Recovery of ²⁴⁴Pu from irradiated targets for production of element 114*

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The ²⁴⁴Pu targets [1] (PuO₂ electrodeposited on Ti backing) were irradiated during recent bombardments with ⁴⁸Ca¹⁰⁺ ions to produce ^{288, 289}114 [2,3]. During these bombardments with up to 3.6 x 10¹⁸ ions, targets and backings underwent changes that made reprocessing and production of new targets for forthcoming experiments desirable.



Figure 1: Teflon vessel.

Recovery of ²⁴⁴Pu from one arc-shaped segment was accomplished as follows: The Al-target frame was inserted into a Teflon vessel containing a cavity into which the frame could be inserted, see Figure 1. With a sharp knife, the target was cut out of the frame, the frame was removed, and the target together with the Ti backing was dissolved in hot conc. HCl. The dissolution of the Ti backing was incomplete. The central part of the backing that had received the highest beam intensity did not dissolve. The resulting solution with the remainder of the undissolved Ti was evaporated to near dryness, transferred into a 10 ml measuring flask and filled with 8 M HCl. An aliquot of that solution was removed, evaporated to dryness, and the α -particle activity was determined. The total activity was used for yield determination. The results indicate that more than 80 % of the Pu had been recovered.

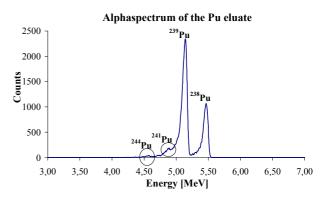


Figure 2: Alphaspectrum of the Pu eluate.

The Pu/Ti solution in 8 M HCl was transferred to a AG 1x8 anion-exchange column (3 x 50 mm), and was washed subsequently with 10 x 1 ml of 8 M HCl to remove the Ti and the ²⁴¹Am from the column. Then, the Pu was eluted from the column in 8 x 1 ml of 0.5 M HCl. Figure 2 shows the spectrum of α particles of an aliquot of the eluate. Due to the isotopic composition of the plutonium (97.9 % 244 Pu, 1.3 % 242 Pu, 0.7 % 240 Pu, < 0.1% other), the main α activities are associated with ^{238}Pu and ²³⁹Pu. 100 µl of that solution was removed, filled up to 2 ml and was irradiated with thermal neutrons in the TRIGA reactor at the Institute of Nuclear Chemistry at the University of Mainz at 100 kW together with a second reference sample containing 9.62 μg ²⁴⁴Pu for 6 h. After a decay time of 18 h, both samples were assayed for the 327.6 keV γ-activity of ²⁴⁵Pu at a Ge detector, see Figure 3. The activation analysis showed a ²⁴⁴Pu recovery of 89

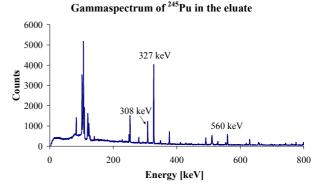


Figure 3: Gammaspectrum of ²⁴⁵Pu solution.

References

- [1] K. Eberhardt *et al.*, contribution to this report.
- [2] Ch. E. Düllmann *et al.*, contribution to this report.
- [3] A. Yakushev *et al.*, contribution to this report.

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